

ENGINEERING CHANGE NOTICE

Page 1 of 21. ECN 631581Proj.
ECN

2. ECN Category (mark one) Supplemental <input type="checkbox"/> Direct Revision <input checked="" type="checkbox"/> Change ECN <input type="checkbox"/> Temporary <input type="checkbox"/> Standby <input type="checkbox"/> Supersedure <input type="checkbox"/> Cancel/Void <input type="checkbox"/>	3. Originator's Name, Organization, MSIN, and Telephone No. Brett C. Simpson, Data Assessment and Interpretation, R2-12, 373-5915		3a. USQ Required? <input type="checkbox"/> Yes <input checked="" type="checkbox"/> No	4. Date 06/25/96
	5. Project Title/No./Work Order No. Tank 241-BX-109		6. Bldg./Sys./Fac. No. 241-BX-109	7. Approval Designator N/A
	8. Document Numbers Changed by this ECN (includes sheet no. and rev.) WHC-SD-WM-ER-572, Rev. 0		9. Related ECN No(s). N/A	10. Related PO No. N/A
11a. Modification Work <input type="checkbox"/> Yes (fill out Blk. 11b) <input checked="" type="checkbox"/> No (NA Blks. 11b, 11c, 11d)	11b. Work Package No. N/A	11c. Modification Work Complete N/A _____ Cog. Engineer Signature & Date		11d. Restored to Original Condition (Temp. or Standby ECN only) N/A _____ Cog. Engineer Signature & Date

12. Description of Change

This ECN is being generated in order to exchange page 5-7. When the document was initially released, pages 5-6 and 5-7 were duplicates.

13a. Justification (mark one)

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13b. Justification Details

Page 5-7 was a duplicate of page 5-6. The correct page 5-7 needed to be placed in the document.

14. Distribution (include name, MSIN, and no. of copies)

See attached distribution.

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Tank Characterization Report for Single-Shell Tank 241-BX-109

Brett C. Simpson

Westinghouse Hanford Company, Richland, WA 99352

U.S. Department of Energy Contract DE-AC06-87RL10930

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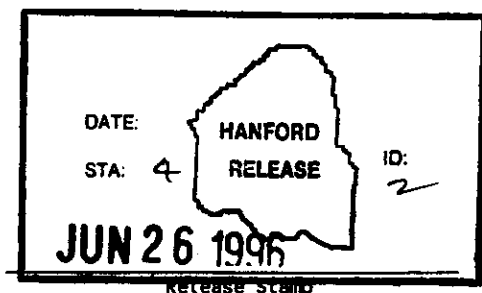
Abstract: N/A

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significantly different from each other. Because each segment is represented by a lower and upper portion, segment level data were analyzed using half-segment resolution. This analysis showed that 16 of the analytes (primarily ICP metals) had P values less than 0.01.

The ANOVA performed on the core composite level revealed that for analytes that contributed substantially to the waste matrix (ie. $> 10,000 \mu\text{g/g}$), very little variability was observed between cores. Relative standard deviations for those analytes were generally less than 10%. They were somewhat higher for analytes present in smaller quantities, but usually not exhibiting extreme variability.

A multivariate cluster analysis was performed on a set of analytes (Cl^- , NO_2^- , NO_3^- , PO_4^{3-} , SO_4^{2-} , Al, Ca, Fe, Mg, Na, Ni, P, S and U). This effort provided two results. It described qualitatively the potential spatial structure within the tank BX-109 waste by identifying similar groups of observations. It also described quantitatively each group or potential waste layer, based on the mean analyte concentration of the associated group of observations. Figure 5-1 illustrates the grouping from results of the clustering analysis. A discussion of the characteristics of each cluster and the cluster analysis approach is included in Appendix E.

In summary, based on the visual descriptions of the samples and the statistical results, the tank contents appear to consist of two distinct layers in the vertical direction (Figure 5-1). The upper layer (segment 1 and the upper half of segment 2) is higher in PO_4 , NO_2 , Al, and Ca, and lower in U concentration than the lower layer. This could be attributed to improvements in uranium recovery processing at U Plant, which resulted in removing more uranium in later years. Composition of the top segment would also be expected to vary due to the tank-to-tank transfers of supernatant in the later years. Very little variability was observed between cores of the composite level. More detailed results of the ANOVA and multivariate cluster analyses are included in Appendix E.

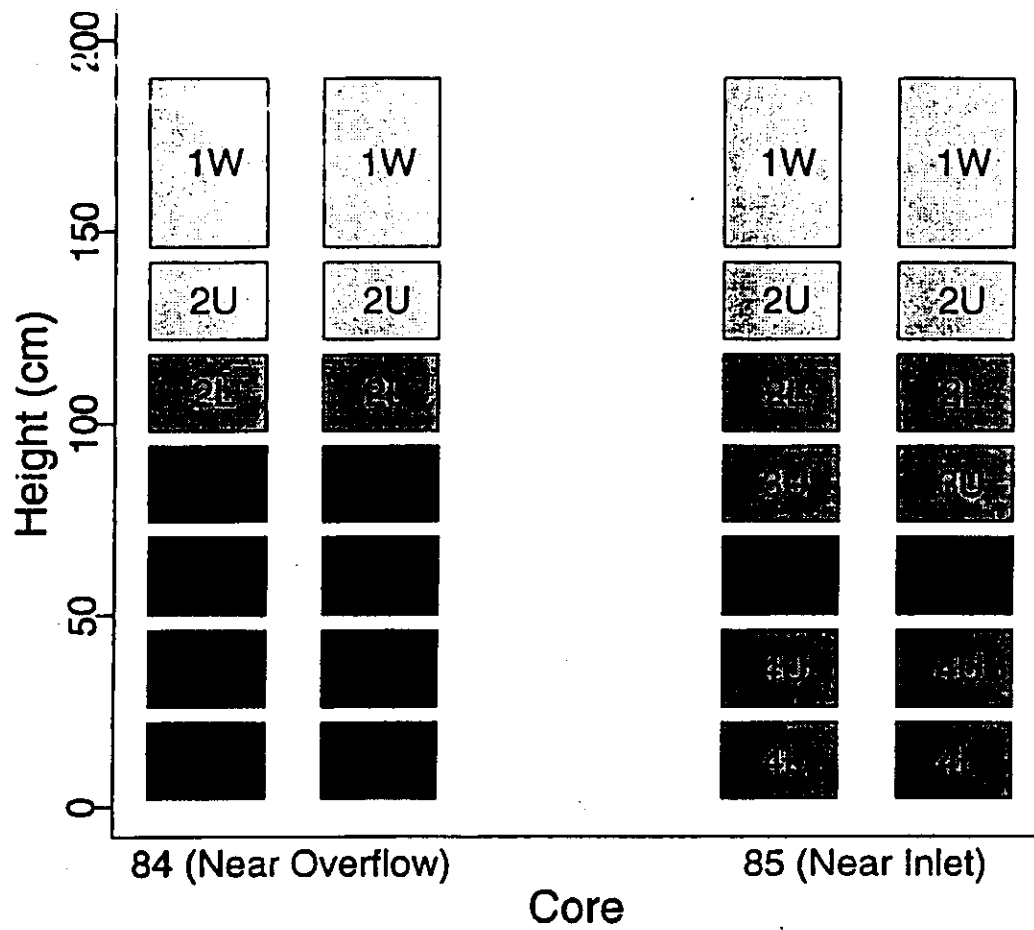
No visual indications or analytical data gathered support the presence of 1C waste in the tank. This information, coupled with the process history data, indicates that the tank contents are entirely UR waste.

5.4 COMPARISON OF TRANSFER HISTORY WITH ANALYTICAL RESULTS

The Hanford Defined Waste (HDW) estimates (Agnew 1996) and TLM estimates for tank 241-BX-109 (Agnew et al. 1996) are compared with the analytical results from the 1995 sampling event in Tables 5-4 and 5-5.

Based on historical tank transfer records, tank 241-BX-109 is expected to contain UR type waste. All historical fingerprint analytes (Na, Fe, H_2O , SO_4 and U) for segment 3 (upper half) were greater than 10% of HDW estimates for a UR waste type and passed the gateway analysis. However, the HDW values were lower than the segment analyses for several of the analytes, especially if water loss through evaporation is considered. HDW estimates were

Figure 5-1. Clustering Results for the BX-109 Cores.
(Refer to color figure in Appendix E, Figure E-3)



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